

## RESEARCH ARTICLE

## BACKGROUND IONIZING RADIATION DOSE LEVELS AND EXCESS LIFETIME CANCER RISK IN ASHAKA CEMENT PLC, ASHAKA, GOMBER STATE, NIGERIA



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## Abstract

**Background:** Radiation monitoring is crucial for assessing environmental safety and public health, particularly in areas with industrial activities such as cement production. Ionizing radiation exposure can have significant health implications, making it essential to evaluate dose levels and associated risks regularly.

**Methods:** This study focuses on the radiation dose levels and excess lifetime cancer risk at multiple locations within Ashaka Cement Plc, situated in Ashaka, Gombe State, Nigeria. By measuring radiation levels and analyzing cancer risk, the research aims to ensure the safety of the environment and the well-being of workers and nearby communities. A Geiger counter rate meter was used to measure the radiation dose level 1 m above the ground level.

**Results:** The results showed that the dose rates ranged from 0.1100 to 0.3467  $\mu\text{Sv/h}$ , with an average of 0.2496  $\mu\text{Sv/h}$ . The mean annual equivalent dose was 0.43728 mSv/y, lower than the 1 mSv/y limit, and the mean annual effective dose was 0.3061 mSv/y, higher than the 0.07 mSv/y, global average but lower than the 1 mSv/y limit. The excess lifetime cancer risk was  $1.53 \times 10^{-3}$ , which is higher than the world average of  $1.45 \times 10^{-3}$ .

**Conclusion:** Generally, background ionizing radiation levels are safe and pose no immediate health risk. Low-level radiation doses may have long-term health effects over a lifetime (70 years), as per excess lifetime cancer risk values.

**Keywords:** *Ashaka Cement, Dose Rate, Effective Dose, Equivalent Dose, Excess Lifetime Cancer Risk.*

## INTRODUCTION

## Background

It is an unavoidable aspect of living on Earth that humans are typically exposed to ionising radiation from natural sources every day (UNSCEAR, 2000). The background radiation that affects humans originates from together natural and artificial sources. Commonly, natural radionuclides of both terrestrial and cosmogenic basis account for 85% of a person's annual total radiation dosage (UNSCEAR, 2000, Belivermis, *et. al.*, 2010). Over the course of life

on Earth, living things have been subjected to radiation from both naturally occurring compounds found in both living and non-living components of the biosphere and radionuclides created by cosmic rays in the atmosphere. Humans are affected by the ionising radiation that is sent into the environment from both manmade and primordial cosmic sources (Essien, *et. al.*, 2017). Ionizing radiation exposure typically results from man-made or natural sources. Many different types of geological formations contain natural radiation sources, such as soils, rocks, plants, water, and air. According to (Abubakar, *et. al.*,

2017). radioactive nuclides from the Earth's crust that make up a significant amount of background radiation are from  $^{238}$  Uranium,  $^{232}$  Thorium, or  $^{40}$  Potassium, with occurrence levels that depend on the geology, altitude, and construction materials.

Radiation that can form ions that can interfere with biological functions is known as ionising radiation. Non-ionizing radiation can have a variety of negative effects on human health, yet it cannot produce ions. Because of the very nature of their surroundings, humans are subject to variable degrees of ambient radiation with or without consent (Farai and Vincent Uchechukwu., 2006).

It is essential to comprehend the effects of radon and its by-products on health. The natural ambient radiation level that originates from the ground, air, water, food, or building materials can be used to calculate the dosage limits of public exposure (Ononugbo, *et. al.*, 2015, El-Taher, 2012). By monitoring the absorbed dose rates in air, one can directly compute the worldwide annual mean exposure to natural radiation sources due to external and internal doses received by a population from background radiation coming from radioisotopes in soils. This is a simple method of determining the influence of exposure levels that humans experience both directly and indirectly through pathways of ingestion and inhalation in various contexts, such as indoor and outdoor spaces. The radionuclide content and activity concentration in the ground, which

vary depending on the kind of bedrock and soil, are related to the outdoor levels (Sara Almgren., 2008, Tayyeb, *et. al.*, 2012 and Moontaha *et. al.*, 2018).

Even at low doses, ionising radiation can induce cancer and genetic disorders. Because of their probabilistic nature and the presumption that any exposure may have an impact, these effects are known as stochastic effects (UNSCEAR, 2000). Furthermore, research has shown that there may be a higher chance of dying from radon exposure indoors (at home or at work) than there is outside, and that there may be a higher estimated risk of lung cancer from radon exposure than there is from all other combined causes (UNSCEAR, 2000). Ionising radiation can alter a material's chemical state, which can have biologically significant effects on the immune system, growth rate, and behavioural fallout from central nervous system injury. High radiation doses have been linked to a number of side effects, including cancer (IAEA, 2005, James et, al., 2015).

#### Study Area

Ashaka Cement Company is located at  $10^{\circ}55'10''$  N and  $11^{\circ}30'00''$ E. The Company was established in 1974 at the initiation of the Nigeria Industrial Development Bank with the encouragement and participation of the Federal Government of Nigeria and north-eastern state governments (Bauchi, Borno, and Gongola State)

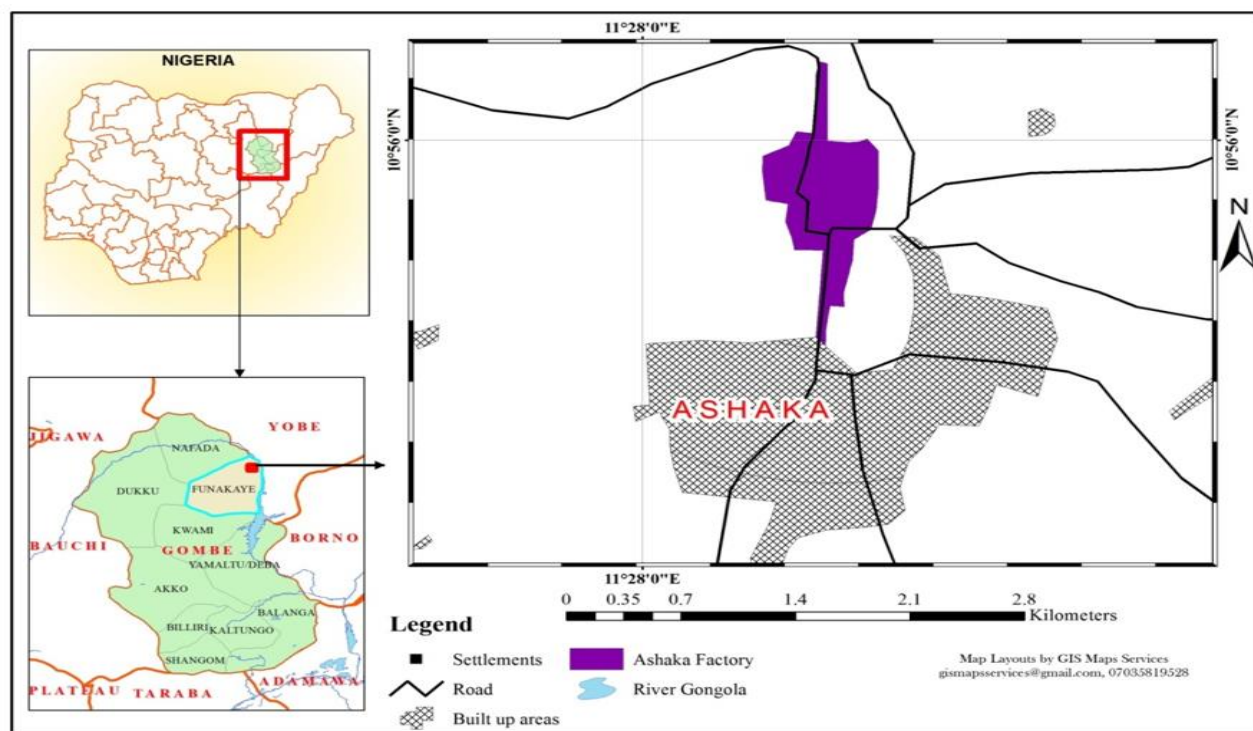


Fig 1 Map showing the location of the study area (Ashaka Cement Company PLC and Ashaka Town)

**Study Aim:**

This study focused on the radiation dose levels and excess lifetime cancer risk at multiple locations within Ashaka Cement Plc, situated in Ashaka, Gombe State, Nigeria

**Materials and Methods:**
**Materials**

The materials utilized for this research include a Geiger counter rate meter, GPS device, measuring tape, pen, and notebook.

**Methods**

A Geiger Counter Rate Meter, Model AP1884-002, was utilized for the detection and measurement of the dosage rate. The radiation dose measuring approach was adopted. Initially, the Geiger counter was calibrated to identify and quantify radiation exposure rate in counts per meter (CPM). Gamma, beta, and X-ray radiation are all monitored by the portable Geiger counter. First, the calibration factor of the device and the meter's zero inaccuracy were confirmed and noted. To prevent contamination or interference on the ground surface, the Geiger counter was positioned at each key location within the study area at a height of one meter (Olanrewaju *et al.*, 2023).

The average values were obtained for each of the fifteen (fifteen) study locations that were selected inside the site after the readings were taken three times at each location. At each point, the total count was recorded for 60 s.

**Estimation of Dose Rate  $DR$  ( $\mu Sv hr^{-1}$ )**

Each count was converted to micro-Sievert per hour ( $\mu Sv hr^{-1}$ ) using Equation 1.

$$1 \text{ CPM} = 0.01 \mu Sv hr^{-1} \dots \dots \dots (1)$$

**Estimation of Annual Equivalent Dose  $A_{equi.}$  ( $mSv y^{-1}$ )**

The annual equivalent dose (AED) was calculated using the formula:

$$\begin{aligned} A_{equi.} (mSv y^{-1}) \\ = \sigma \times \mu \times 24 \times 365 \\ \times 10^{-3} \dots \dots \dots (2) \end{aligned}$$

Where  $\sigma = (\mu Sv hr^{-1})$ ,  $\mu = 0.2$  (outdoor occupancy factor) (UNSCEAR, 2000).

**Annual Effective Dose  $A_{eff.}$  ( $mSv y^{-1}$ ) and External Dose  $ED$  ( $nGy hr^{-1}$ )**

In this study, the annual effective dose is calculated by employing the outdoor external dose, occupancy factor, or proportion of the total outdoor time that an individual is exposed to the radiation  $\mu = 0.2$  of  $8760 \text{ hr}$  within a year, and the conversion factor ( $CF$ ) =  $0.7 (Sv Gy^{-1})$  for converting the absorbed dose in air to an effective dose.

The equations given by UNSCEAR, 2000 and Qureshi *et al.* 2014 are as follows:

$$\begin{aligned} A_{eff.} (mSv y^{-1}) = ED (nGy hr^{-1}) \times 0.2 \times 8760 \text{ hr} \\ \times 0.7 Sv Gy^{-1} \times 10^{-3} \dots \dots \dots (3) \end{aligned}$$

Where

$$\begin{aligned} ED (nGy hr^{-1}) \\ = \frac{DR (\mu Sv hr^{-1})}{Q} \times 10^{-3} \dots \dots \dots (4) \end{aligned}$$

Where Q is equal to 1

**Estimation of Excess Lifetime Cancer Risk (ELCR)**

To assess the cancer risk of the workers resulting from BIR exposure, excess lifetime cancer risk (ELCR) was estimated by using equation (5) Abdulkareem *et al.* 2023.

$$\begin{aligned} ELCR \\ = A_{equi.} \times DL \\ \times RF \dots \dots \dots (5) \end{aligned}$$

$A_{equi.}$  Is the Annual Equivalent Dose, DL is the average Duration of Life and is approximately equal to **70 years** RF, is the Risk Factor or fatal cancer risk measured in per sievert ( $Sv^{-1}$ ). For stochastic effects from low-dose background radiation, ICRP 2007. A value of 0.05 for public exposure.

**Results:**

The background ionizing radiation and excess lifetime cancer risk were estimated from fifteen (15) different locations and the results were obtained using the Geiger Muller counter, the Annual Equivalent Dose ( $A_{equi.}$ ), External dose (ED), and annual effective

dose ( $A_{eff.}$ ) and Excess Lifetime Cancer Risk (ELCR) for each location with a Global Positioning System (GPS) were determined and are presented in Table 1.

Table 1: Dose Rate ( $\mu\text{Sv/h.}$ ), Annual Equivalent Dose ( $\text{mSv/y}$ ), External Dose ( $\text{nGy/h.}$ ), Annual Effective Dose ( $\text{mSv/y}$ ) and Excess Lifetime Cancer Risk Rate ( $\text{ELCR} \times 10^{-3}$ )

| Location                 | Latitude | Longitude | CPM | DR ( $\mu\text{Sv/h.}$ ) | $A_{equi.}$ ( $\text{mSv/y}$ ) | ED ( $\text{nGy/h.}$ ) | $A_{eff.}$ ( $\text{mSv/y}$ ) | ELCR X $10^{-3}$ |
|--------------------------|----------|-----------|-----|--------------------------|--------------------------------|------------------------|-------------------------------|------------------|
| Kiln Site                | 10.9308  | 11.4781   | 24  | 0.2400                   | 0.4205                         | 0.00024                | 0.2943                        | 1.4717           |
| Coal Pack                | 10.9322  | 11.4792   | 19  | 0.1900                   | 0.3329                         | 0.00019                | 0.2330                        | 1.1651           |
| Coal Mill                | 10.9308  | 11.4800   | 32  | 0.3200                   | 0.5606                         | 0.00032                | 0.3924                        | 1.9622           |
| Training Scholl Workshop | 10.9306  | 11.4281   | 23  | 0.2267                   | 0.3972                         | 0.00023                | 0.2780                        | 1.3901           |
| Crusher                  | 10.9322  | 11.4778   | 34  | 0.3400                   | 0.5957                         | 0.00034                | 0.4170                        | 2.0849           |
| Reclaimer                | 10.9322  | 11.4972   | 28  | 0.2800                   | 0.4906                         | 0.00028                | 0.3434                        | 1.7170           |
| Raw Mill                 | 10.9319  | 11.4803   | 24  | 0.2430                   | 0.4257                         | 0.00024                | 0.2980                        | 1.4901           |
| Coal Power Plant Area    | 10.9308  | 11.4775   | 35  | 0.3467                   | 0.6074                         | 0.00035                | 0.4252                        | 2.1260           |
| Power house (New)        | 10.9325  | 11.4603   | 11  | 0.1100                   | 0.1927                         | 0.00011                | 0.1349                        | 0.6745           |
| Power House (Old)        | 10.9328  | 11.4442   | 16  | 0.1600                   | 0.2803                         | 0.00016                | 0.1962                        | 0.9811           |
| Cement Mill              | 10.9308  | 11.4778   | 26  | 0.2633                   | 0.4613                         | 0.00026                | 0.3229                        | 1.6146           |
| Gypsum Pack              | 10.9308  | 11.4781   | 30  | 0.3033                   | 0.5314                         | 0.00030                | 0.3720                        | 1.8598           |
| Raw mill Silo            | 10.9311  | 11.4775   | 28  | 0.2767                   | 0.4848                         | 0.00028                | 0.3393                        | 1.6967           |
| Compressor Room          | 10.9317  | 11.4947   | 18  | 0.1833                   | 0.3211                         | 0.00018                | 0.2248                        | 1.1240           |
| Parking Plant            | 10.9333  | 11.5058   | 30  | 0.3033                   | 0.5314                         | 0.00030                | 0.3720                        | 1.8598           |

## Discussion:

Table 1 presents the results of the radiation index parameters. The Dose Rate mean value varied between  $0.1100 \mu\text{Sv/h}$  and  $0.3467 \mu\text{Sv/h}$ . The power house (new) recorded the lowest value of  $0.1100 \mu\text{Sv/h}$ , while the coal power plant area yielded the highest value of  $0.3467 \mu\text{Sv/h}$ . The Annual Equivalent Dose has a mean value of  $0.1927 \text{mSv/h}$  and a maximum value of  $0.5957 \text{mSv/h}$  reported from the crusher. The minimum value was  $0.1927 \text{mSv/h}$  recorded from the power house (new). The range of external dose rates is  $0.00011 \text{nGy/h}$  to  $0.00034 \text{nGy/h}$ . The minimum value of  $0.00011 \text{nGy/h}$  was recorded from the power house (new), and the maximum value of  $0.00034 \text{nGy/h}$  was from the crust. The mean value of the annual effective dose ranged from  $0.1349 \text{mSv/y}$  to  $0.4252 \text{mSv/y}$ ; the minimum value of  $0.1349 \text{mSv/y}$  was recorded from the power house (new), and the maximum value of  $0.4252 \text{mSv/y}$  was recorded from the coal power plant area. All of these values fall below the suggested allowable levels, which are  $1.0 \text{mSv/y}$  for the general population and  $20.0 \text{mSv/y}$  for occupational workers. ICRP 2007.

The mean value of the excess lifetime cancer risk ranged from  $0.6745 \times 10^{-3}$  to  $2.0849 \times 10^{-3}$ , the power plant (New) recorded the lowest value of  $0.6745 \times 10^{-3}$ , while the coal plant area recorded the

highest value of  $2.0849 \times 10^{-3}$ . The United Nations Scientific Committee on the Effects of Atomic Radiation UNSCEAR 2000, has established that the obtained ELCR values are more than the global average, which is  $0.29 \times 10^{-3}$ . These numbers represent the likelihood of developing cancer throughout a 70-year lifespan.

The results obtained for the radiation dose rate in this study are quite higher than the results obtained by Achuka et al., 2019. Though there is no evidence of radioactive element occurrences, the likelihood of workers and study area residents developing cancer in this environment can be used to compute excess lifetime cancer risk (ELCR). Conferring to the Linear No Threshold (LNT) hypothesis, extrapolation from indication continuous, high-dose reactions to low-dose responses, all acute ionizing radiation exposures down to zero are harmful Sunday et. al., 2017. Establish that the radiation dose rate was found to be  $0.133 \text{Sv/h}$  for the general public was not even close to being reached at any of the locations' indoor dose rate levels. As a result, the radiation exposure rate values in these particular departments may not suggest any possible risk to staff members or the general public. These outcomes can be used as a guide when determining the indoor background radiation levels in different places. It was determined that  $2.86 \times 10^{-3}$  is the excess lifetime cancer risk factor for the research location. That is more than the global average and higher than our finding. This is most likely

produced by the rock and other building materials used, as well as the lab chemicals and medications transmitted to the new location.

### Conclusion

The background ionising radiation levels and excess lifetime cancer risk were analysed in fifteen (15) distinct locations inside Ashaka Cement PLC, although the Excess Lifetime Cancer Risk (ELCR) was greater than the permissible limit, all projected radiation index parameters were within the ranges advised for both employees and the general public.

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### Competing Interests

The author declares that there are no conflicts of interest related to this study.

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